

Unstructured FEM Neutron Transport William C. Dawn

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## Unstructured Finite Element Neutron Transport using MFEM

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## Microreactors

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- Relatively small electrical output (<10 MW).
- Geometrically compact.
- Varied designs.
  - ► Thermal, epithermal, and fast neutron spectra.
  - Unique cooling designs (e.g., heat pipes).
  - Space applications.

# NC STATE MARVEL Microreactor

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- Development led by Idaho National Laboratory (INL) for construction on-site.
- 18 kW to 25 kW electric output.
- Cylinder with 50 cm height and 120 cm diameter.
- UZrH HALEU fuel.
- Thermal spectrum.
- Liquid NaK eutectic coolant.





# Why MFEM?

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- Exascale Computing Project (ECP).
- GPU support.
- Library support (e.g., HYPRE, PETSc, SLEPc, AmgX, etc.).
- Rapid prototyping.
  - Essential for research computer programs.
  - Dozens of solver methods implemented.

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2021			



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# **NC STATE** Neutron Transport Equation

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- Steady-state Boltzmann equation.
- 6 independent variables:
  - ► 3 space.
  - 2 direction (angle).
  - 1 energy.
- Eigenvalue problem for eigenvalue  $\lambda$  and eigenfunction  $\psi(\mathbf{r}, \hat{\Omega}, E)$ .
- Coefficients are "cross sections." Interaction probabilities.

$$\begin{split} \hat{\Omega} \cdot \nabla \psi(\mathbf{r}, \hat{\Omega}, E) + \Sigma_t(\mathbf{r}, E) \psi(\mathbf{r}, \hat{\Omega}, E) &= \\ & \frac{\chi(\mathbf{r}, E)}{\lambda} \int_0^\infty \nu \Sigma_f(\mathbf{r}, E') \int_{4\pi} \psi(\mathbf{r}, \hat{\Omega}', E') \ d\hat{\Omega}' + \\ & \int_0^\infty \int_{4\pi} \Sigma_s(\mathbf{r}, \hat{\Omega}' \cdot \hat{\Omega}, E' \to E) \psi(\mathbf{r}, \hat{\Omega}', E') \ d\hat{\Omega}' \ dE' \end{split}$$

# Neutron Transport Equation – Multigroup Discretization

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- Multigroup approximation in energy (e.g., 24 groups).
- Multigroup constants conserve reaction rates.

$$\begin{split} \hat{\Omega} \cdot \nabla \psi_{g}(\mathbf{r}, \hat{\Omega}) + \Sigma_{t,g}(\mathbf{r}) \psi_{g}(\mathbf{r}, \hat{\Omega}) &= \\ \frac{\chi_{g}(\mathbf{r})}{\lambda} \sum_{g'=1}^{N_{G}} \nu \Sigma_{f,g'}(\mathbf{r}) \int_{4\pi} \psi_{g'}(\mathbf{r}, \hat{\Omega}') \ d\hat{\Omega}' + \sum_{g'=1}^{N_{G}} \int_{4\pi} \Sigma_{s,g' \to g}(\mathbf{r}, \hat{\Omega}' \cdot \hat{\Omega}) \psi_{g'}(\mathbf{r}, \hat{\Omega}') \ d\hat{\Omega}' \end{split}$$

$$\psi_g(\mathbf{r},\hat{\Omega}) \equiv \int_{E_g}^{E_{g-1}} \psi(\mathbf{r},\hat{\Omega},E) \ dE$$

## Neutron Transport Equation – Discrete Ordinates Discretization

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- Discrete ordinates  $(S_N)$  approximation in angle.
- Full derivation projects  $\Sigma_s$  onto spherical harmonics.

$$N_A = \begin{cases} N(N+2) & 3D\\ \frac{N(N+2)}{2} & 2D \end{cases}$$



 $\hat{\Omega}_{n} \cdot \nabla \psi_{g,n}(\mathbf{r}) + \Sigma_{t,g}(\mathbf{r})\psi_{g,n}(\mathbf{r}) =$ Level-symmetric S<sub>10</sub> quadrature.  $\frac{\chi_{g}(\mathbf{r})}{\lambda} \sum_{g'=1}^{N_{G}} \nu \Sigma_{f,g'}(\mathbf{r}) \sum_{n'=1}^{N_{A}} w_{n}\psi_{g',n'}(\mathbf{r}) + \sum_{g'=1}^{N_{G}} \sum_{n'=1}^{N_{A}} S_{g' \to g,n' \to n}(\mathbf{r})\psi_{g',n'}(\mathbf{r})$ 

## Vector Dimension

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- Number of unknowns per spatial grid point.
- Unique to neutron transport due to underlying phase space.
- High-dimensional continuous phase space leads to high-dimensional discrete phase space.
- Example:
  - ▶ 24 group, S<sub>10</sub> in 2D.
  - ▶  $N_G = 24$ ,  $N_A = 60$ . VDIM = 1440.
- Most DOF are due to energy & angle (not space).
- Linear finite elements. Spectral FEM likely not useful.

# **NC STATE** SAAF Equations for FEM

Self-Adjoint Angular Flux (SAAF).

Second-order system of PDEs can be solved with continuous FEM.

• Convert hyperbolic to elliptic.

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No sweep.  $\int_{\Omega} \frac{1}{\sum_{t=0}^{t} (\mathbf{r})} \hat{\Omega}_{n} \cdot \nabla v(\mathbf{r}) \hat{\Omega}_{n} \cdot \nabla \psi_{g,n}(\mathbf{r}) \, d\mathbf{r} + \int_{\Omega} v(\mathbf{r}) \sum_{t,g} (\mathbf{r}) \psi_{g,n}(\mathbf{r}) \, d\mathbf{r} +$  $\frac{1}{2} \left( \oint_{\Gamma} v(\mathbf{r}) \psi_{g,n}(\mathbf{r}) \hat{\Omega}_{n} \cdot \hat{\mathbf{n}} \, d\Gamma + \oint_{\Gamma} v(\mathbf{r}) \psi_{g,n}(\mathbf{r}) \left| \hat{\Omega}_{n} \cdot \hat{\mathbf{n}} \right| \, d\Gamma \right) =$  $\left|\frac{1}{\lambda}\left(\int_{\mathcal{D}}\chi_{g}(\mathbf{r})\sum_{r=l-1}^{N_{G}}\nu\Sigma_{f,g'}(\mathbf{r})\sum_{r=l-1}^{N_{A}}w_{n'}\psi_{g',n'}(\mathbf{r})\,d\mathbf{r}+\int_{\mathcal{D}}\frac{\hat{\Omega}_{n}\cdot\nabla\nu(\mathbf{r})}{\Sigma_{t,g}(\mathbf{r})}\chi_{g}(\mathbf{r})\sum_{r=l-1}^{N_{G}}\nu\Sigma_{f,g'}(\mathbf{r})\sum_{r=l-1}^{N_{A}}w_{n'}\psi_{g',n'}(\mathbf{r})\,d\mathbf{r}\right|$  $\int_{\mathcal{D}} v(\mathbf{r}) \sum_{\ell=1}^{N_G} \sum_{\ell=1}^{N_A} S_{g' \to g, n' \to n}(\mathbf{r}) \psi_{g', n'}(\mathbf{r}) \, d\mathbf{r} + \int_{\mathcal{D}} \frac{\hat{\Omega}_n \cdot \nabla v(\mathbf{r})}{\Sigma_{\ell,g}(\mathbf{r})} \sum_{\ell=1}^{N_G} \sum_{\ell=1}^{N_A} S_{g' \to g, n' \to n}(\mathbf{r}) \psi_{g', n'}(\mathbf{r}) \, d\mathbf{r} \frac{1}{2} \left( \oint_{\Gamma} v(\mathbf{r}) \psi_{g,n}^{\text{inc}}(\mathbf{r}) \hat{\Omega}_n \cdot \hat{\mathbf{n}} \, d\Gamma - \oint_{\Gamma} v(\mathbf{r}) \psi_{g,n}^{\text{inc}}(\mathbf{r}) \left| \hat{\Omega}_n \cdot \hat{\mathbf{n}} \right| \, d\Gamma \right)$ 

# Solver Methodologies

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## Generalized Eigensolver.

- All-at-once.
- PETSc + SLEPc.
  - Generalized Davidson.

## • Approx. 100 GMRES iteration preconditioning. • Scattering iterations with Diffusion Synthetic

- NVIDIA AmgX library.
- HYPRE+GPU in development.

## Challenge:

Memory limited.

# • Approx. 10 000 PCG solves.

• HyprePCG + HypreBoomerAMG.

• One group at-a-time (all angle).

Gauss-Seidel in energy.

• "Lagged" source term.

Acceleration (DSA).

Source Iteration.



# Source Iteration Algorithm



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Increasing "Matrix-free"



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## MARVEL Reactor Model

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- Series of two-dimensional meshes generated with Gmsh.
- 24 and 238 group cross sections generated with data from SCALE.
- All results use 24 group structure.
- Large gaseous region (approx. 12 % by volume).
- Gas density artificially increased 100 ×.

Refinement	$h_{\text{inner}}  [\text{cm}]$	$h_{\text{outer}}  [\text{cm}]$	Elements	Vertices
R0	0.5	1.0	45 204	22 791
R1	0.25	0.5	439 168	220 333
R2	0.125	0.25	6322912	3 164 449
R3	0.0625	0.125	96 378 000	48 200 969

### **NC STATE** UNIVERSITY MARVEL Reactor Geometry





# **NC STATE** MARVEL Reactor Results

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- Source iteration solver necessary.
- Most refined results require 256 Summit nodes (10752 MPI ranks) and 6 h.
- More spatially refined results desirable.
- Observed convergence rate  $O(h^{1.340})$ .

Refinement	$S_2$	$S_4$	S <sub>6</sub>	$S_8$	$S_{10}$
<b>R</b> 0	1.325 219	1.326 908	1.326742	1.326 872	1.327 017
R1	1.335 505	1.336 675	1.336745	1.336790	1.336 872
R2	1.339 502	1.340 601	1.340697		
Ref. <sup>†</sup>	1.342042	1.343240	1.343278		

<sup>†</sup> Richardson extrapolation.

Refinemen	t S <sub>2</sub>	$S_4$	S <sub>6</sub>	S <sub>8</sub>	S <sub>10</sub>
R0	2.19 M	6.56 M	13.13 M	21.88 M	32.82 M
R1	21.15 M	63.46 M	126.91 M	211.52 M	317.28 M
R2	303.79 M	911.36 M	1.82 B	*	

# **NC STATE** MARVEL Reactor Results

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Thermal scalar flux. E < 0.625 eV Epithermal scalar flux. 0.625 eV < E < 6 keV Fast scalar flux. E > 6 keV

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## Mesh Refinement Effects

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Two entirely separate effects:

- 1. Better describe curved geometry.
- 2. Decrease characteristic mesh size.
- 1.64 % fuel volume error for R0 mesh.
- Brief investigation of "mesh correction" methods.
  - ► Equidistant vertices.
  - Volume preserving meshes.
  - Cross section correction.
- Eigenvalue change due to refinement is mostly attributable to improved mass conservation.

Refinement	Corrected	Uncorrected	Difference [pcm]
R0 R1	1.334 091 1.338 354	1.326 872 1.336 790	721.88 156.41
	MARVE	L 2D S <sub>8</sub> Resul	ts.



# MARVEL 3D Model

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# Next Steps

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- Improved GPU utilization.
  - ► HYPRE+GPU.
  - "Partial"/tensor assembly of linear forms on GPUs.
- "Large" meshes.
- Heat conduction multiphysics feedback.
- Demonstration of three-dimensional results.

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## Thank You!

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Thank you all for your attention this afternoon!

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